

November 7, 2005

Mr. Christopher M. Crane
President and Chief Nuclear Officer
Exelon Nuclear
Exelon Generation Company, LLC
Quad Cities Nuclear Power Station
4300 Winfield Road
Warrenville, IL 60555

SUBJECT: QUAD CITIES NUCLEAR POWER STATION, UNITS 1 AND 2
NRC EVALUATION OF CHANGES, TESTS, OR EXPERIMENTS AND
PERMANENT PLANT MODIFICATIONS BASELINE INSPECTION REPORT
05000254/2005007; 05000265/2005007(DRS)

Dear Mr. Crane:

On October 7, 2005, the U.S. Nuclear Regulatory Commission (NRC) completed a combined baseline inspection of the Evaluation of Changes, Tests, or Experiments and Permanent Plant Modifications at the Quad Cities Nuclear Power Station, Units 1 and 2. The enclosed report documents the results of the inspection, which were discussed with Mr. M. Perito and others of your staff at the completion of the inspection on October 7, 2005.

The inspectors examined activities conducted under your license as they relate to safety and compliance with the Commission's rules and regulations and with the conditions of your license. The inspectors reviewed selected procedures and records, observed activities, and interviewed personnel. Based on the results of the inspection no findings were identified.

In accordance with 10 CFR 2.390 of the NRC's "Rules of Practice," a copy of this letter and its enclosure will be available electronically for public inspection in the NRC Public Document Room or from the Publicly Available Records (PARS) component of NRC's document system (ADAMS). ADAMS is accessible from the NRC Web site at <http://www.nrc.gov/reading-rm/adams.html> (the Public Electronic Reading Room).

Sincerely,

/RA/

David E. Hills, Chief
Engineering Branch 1
Division of Reactor Safety

Docket Nos. 50-254; 50-265
License Nos. DPR-29; DPR-30

Enclosure: Inspection Report 05000263/2005011(DRS)

See Attached Distribution

cc w/encl: Site Vice President - Quad Cities Nuclear Power Station
Plant Manager - Quad Cities Nuclear Power Station
Regulatory Assurance Manager - Quad Cities Nuclear Power Station
Chief Operating Officer
Senior Vice President - Nuclear Services
Senior Vice President - Mid-West Regional
Operating Group
Vice President - Mid-West Operations Support
Vice President - Licensing and Regulatory Affairs
Director Licensing - Mid-West Regional
Operating Group
Manager Licensing - Dresden and Quad Cities
Senior Counsel, Nuclear, Mid-West Regional
Operating Group
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Vice President - Law and Regulatory Affairs
Mid American Energy Company
Assistant Attorney General
Illinois Emergency Management Agency
State Liaison Officer, State of Illinois
State Liaison Officer, State of Iowa
Chairman, Illinois Commerce Commission
D. Tubbs, Manager of Nuclear
MidAmerican Energy Company

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 Engineering Branch 1
 Division of Reactor Safety

Docket Nos. 50-254; 50-265
 License Nos. DPR-29; DPR-30

Enclosure: Inspection Report 05000263/2005011(DRS)

See Attached Distribution

See Previous Concurrence

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DATE	11/03/05	11/03/05	11/07/05		

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cc w/encl: Site Vice President - Quad Cities Nuclear Power Station
Plant Manager - Quad Cities Nuclear Power Station
Regulatory Assurance Manager - Quad Cities Nuclear Power Station
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Senior Vice President - Nuclear Services
Senior Vice President - Mid-West Regional
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Document Control Desk - Licensing
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Mid American Energy Company
Assistant Attorney General
Illinois Emergency Management Agency
State Liaison Officer, State of Illinois
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U.S. NUCLEAR REGULATORY COMMISSION

REGION III

Docket No: 50-254; 50-265
License No: DPR-29; DPR-30

Report No: 05000254/2005007; 05000265/2005007(DRS)

Licensee: Exelon Generation Company, LLC

Facility: Quad Cities Nuclear Power Station, Units 1 and 2

Location: **22710 206th Avenue North
Cordova, IL 61242**

Dates: October 3 through 7, 2005

Inspectors: R. Daley, Senior Reactor Inspector, Team Leader
D. Schrum, Reactor Inspector
N. Valos, Reactor Inspector

Approved by: D. Hills, Chief
Engineering Branch 1
Division of Reactor Safety (DRS)

SUMMARY OF FINDINGS

IR 05000254/2005007; 05000265/2005007(DRS); 10/03/2005 - 10/07/2005; Quad Cities Nuclear Power Station, Units 1 and 2; Evaluation of Changes, Tests, or Experiments (10 CFR 50.59) and Permanent Plant Modifications.

The inspection covered a one-week announced baseline inspection on evaluations of changes, tests or experiments and permanent plant modifications. The inspection was conducted by three regional based engineering inspectors. Two Unresolved Items were identified. The significance of most findings is indicated by their color (Green, White, Yellow, Red) using Inspection Manual Chapter 0609, "Significance Determination Process (SDP)." Findings for which the SDP does not apply may be Green, or be assigned a severity level after NRC management review. The NRC's program for overseeing the safe operation of commercial nuclear power reactors is described in NUREG-1649, "Reactor Oversight Process," Revision 3, dated July 2000.

A. Inspector-Identified and Self-Revealed Findings

No findings of significance were identified.

B. Licensee-Identified Violations

No findings of significance were identified.

REPORT DETAILS

1. REACTOR SAFETY

Cornerstones: Initiating Events, Mitigating Systems, and Barrier Integrity

1R02 Evaluations of Changes, Tests, or Experiments (71111.02)

.1 Review of 10 CFR 50.59 Evaluations and Screenings

a. Inspection Scope

From October 3 through 7, 2005, the inspectors reviewed six evaluations performed pursuant to 10 CFR 50.59. The inspectors confirmed that the evaluations were thorough and that prior NRC approval was obtained as appropriate. The inspectors also reviewed 17 screenings where licensee personnel had determined that a 10 CFR 50.59 evaluation was not necessary. In regard to the changes reviewed where no 10 CFR 50.59 evaluation was performed, the inspectors verified that the changes did not meet the threshold to require a 10 CFR 50.59 evaluation. The evaluations and screenings were chosen based on risk significance, safety significance, and complexity. The list of documents reviewed by the inspectors is included as an attachment to this report.

The inspectors used, in part, Nuclear Energy Institute (NEI) 96-07, "Guidelines for 10 CFR 50.59 Implementation," Revision 1, to determine acceptability of the completed evaluations and screenings. The NEI document was endorsed by the NRC in Regulatory Guide 1.187, "Guidance for Implementation of 10 CFR 50.59, Changes, Tests, and Experiments," dated November 2000. The inspectors also consulted Part 9900 of the NRC Inspection Manual, "10 CFR Guidance for 10 CFR 50.59, Changes, Tests, and Experiments."

b. Findings

b.1 Downgrade of Relief Valves from Category I Environmental Qualification (EQ) to Category II EQ Components

During the inspection, the team identified that in 2004 the licensee had replaced the Unit 2 reactor pressure vessel relief valves, Target Rock Power Operated Relief Valves (PORVs) qualified Category I in accordance with the EQ requirements in 10 CFR 50.49, with Dresser Electromatic Relief Valves (ERVs) qualified as Category II EQ Components. The inspectors determined that this EQ downgrading of the reactor pressure vessel relief valves appeared to be in violation of the requirements in 10 CFR 50.49, "Environmental Qualification of Electric Equipment Important to Safety for Nuclear Power Plants."

The licensee originally replaced the Unit 2 reactor pressure vessel relief valves in 1995. The old Dresser ERVs - the original design for the reactor pressure vessel relief valves - were replaced by Target Rock PORVs. Consistent with the provisions of 10 CFR 50.49, the replacement valves were EQ upgraded in accordance with the Category I

requirements specified in NUREG-0588, "Interim Staff Position on Environmental Qualification of Safety-Related Electric Equipment."

After several years of operation, the licensee decided to replace the PORVs because of operational performance issues with these replacement valves. In 2004, the licensee replaced the PORVs in Unit 2 with the ERVs that were in the plant's initial design. The licensee justified the change because the ERVs were within the plant's original license and design basis. However, even though the licensee purchased new ERVs to replace the PORVs, these valves still were not EQ qualified to Category 1 unlike the valves that they replaced.

The licensee justified replacing the PORVs with the non-Category 1 ERVs by performing a "Sound Reasons to the Contrary" evaluation. 10 CFR 50.49(l) requires that replacement equipment be qualified in accordance with the provisions of 10 CFR 50.49 unless there are sound reasons to the contrary. Regulatory Guide 1.89, "Environmental Qualification of Certain Electric Equipment Important to Safety for Nuclear Power Plants," contains guidance on the justifications that would qualify as "sound reasons to the contrary." Based upon this guidance, the licensee, in EC 345004, Revision 0, reasoned that "a suitable replacement design, qualified in accordance with 10 CFR 50.49, would require significant plant modifications to accommodate its use." This reasoning was primarily based upon the cost of the next cheapest alternative (a Crosby direct acting valve). The cost of these replacement valves would be greater than the cost of reverting to the original ERV design.

The inspectors questioned whether this reversion to the original design was allowable under 10 CFR 50.49. Specifically, since the PORVs were already established as part of the design and licensing basis of the plant, the inspection team questioned if it was allowable under 10 CFR 50.49 and whether the licensee's "Sound Reasons to the Contrary" were adequate. This issue is being treated as an Unresolved Item pending further evaluation of the requirements for qualification of replacement equipment in 10 CFR 50.49. (URI 05000254/2005007-01; URI 05000265/2005007-01)

1R17 Permanent Plant Modifications (71111.17B)

a. Inspection Scope

From October 3 through 7, 2005, the inspectors reviewed six permanent plant modifications that had been installed in the plant during the last two years. The modifications were chosen based upon risk significance, safety significance, and complexity. As per inspection procedure 71111.17B, one modification was chosen that affected the barrier integrity cornerstone. The inspectors reviewed the modifications to verify that the completed design changes were in accordance with the specified design requirements and the licensing bases and to confirm that the changes did not adversely affect any systems' safety function. Design and post-modification testing aspects were verified to ensure the functionality of the modification, its associated system, and any support systems. The inspectors also verified that the modifications performed did not place the plant in an increased risk configuration.

The inspectors also used applicable industry standards to evaluate acceptability of the modifications. The list of modifications and other documents reviewed by the inspectors is included as an attachment to this report.

b. Findings

b.1 Lack of a Design Analysis Evaluating Secondary Fire Effects of Non-Fused 120 VAC Control Circuitry on the Plants' Fire Protection Safe Shutdown Analysis

The inspection team identified that 120 VAC circuitry did not have fusing to isolate potential faults in the circuitry, nor did the licensee have a design analysis to support this configuration during fire scenarios. The inspectors were concerned that these unfused circuits could cause secondary fires at the station should faults occur in the control cabling.

During inspection related to modification EC 336482, "Reconfigure the Motor Control Circuit for the 2B SBLC Pump," the team discovered that in certain configurations of 120 VAC circuitry, no fuse isolation existed. The primary configuration of concern was 120 VAC ungrounded circuits powered from Control Power Transformers (CPTs)(480 - 120 VAC). Based upon this information, the inspectors asked if the licensee's engineering staff had considered the potential adverse effects on the plant's fire protection safe shutdown analysis. Specifically, the inspectors were concerned that if a fire were to occur in a fire area and cause a fault on one of these unfused 120 VAC circuits, a concurrent fire could occur somewhere else in the circuitry due to the high amperage caused by the faulted condition. This could adversely affect the safe shutdown functions during a fire, because it is an implicit assumption that only one fire, in one fire area, can occur at one time. This type of condition invalidates this assumption.

While the licensee was able to produce an analysis that addressed possible grounding and/or shorting out of the CPTs, the licensee was not able to produce any analysis which addressed the possible faulting of the control cabling in other areas. The engineering staff was able to produce an interoffice memorandum between their Architect Engineering firm, Sargent & Lundy, and the licensee dated February 11, 1985 that pertained to this issue; however, this document appeared to only address potential fires from the CPTs.

While it is expected that in most circuits, the licensee will be able to show that the CPTs would fail before the cabling, it is uncertain if, and how many, cases exist where the cabling has the potential to fault in other areas prior to the CPT failing. At the time of the inspection, the licensee could not provide such an evaluation. To address this issue, the licensee initiated corrective action document AR 00382847. Since the licensee had still not been able to conclusively determine whether this issue adversely affected the licensee's fire protection safe shutdown analysis, this issue was considered an Unresolved Item pending a full evaluation by the licensee of this circuitry configuration. (URI 05000254/2005007-02; URI 05000265/2005007-02)

4. OTHER ACTIVITIES (OA)

4OA2 Identification and Resolution of Problems

.1 Routine Review of Condition Reports

a. Inspection Scope

From October 3 through 7, 2005, the inspectors **reviewed nine Corrective** Action Process documents (CAPs) that identified or were related to 50.59 evaluations and permanent plant modifications. The inspectors reviewed these documents to evaluate the effectiveness of corrective actions related to permanent plant modifications and evaluations for changes, tests, or experiments issues. In addition, corrective action documents written on issues identified during the inspection were reviewed to verify adequate problem identification and incorporation of the problems into the corrective action system. The specific corrective action documents that were sampled and reviewed by the team are listed in the attachment to this report.

b. Findings

No findings of significance were identified.

4. OTHER ACTIVITIES

4OA6 Meetings

.1 Exit Meeting

The inspectors presented the inspection results to Mr. M. Perito and others of the licensee's staff, on October 7, 2005. Licensee personnel acknowledged the inspection results presented. Licensee personnel were asked to identify any documents, materials, or information provided during the inspection that were considered proprietary. No proprietary information was identified.

ATTACHMENT: SUPPLEMENTAL INFORMATION

SUPPLEMENTAL INFORMATION

KEY POINTS OF CONTACT

Licensee

W. Beck, Regulatory Assurance Manager
B. Edmark, Design Engineer
R. Gideon, Plant Manager
K. Moser, Site Engineering Director
M. Perito, Operations Director
W. Porter, Design Engineering Manager
J. Taft, Electrical Engineering Supervisor
M. Wagner, Regulatory Assurance Engineer

Nuclear Regulatory Commission

G. Dick, NRR Project Manager
D. Hills, Chief, Engineering Branch 1
M. Kurth, Resident Inspector
M. Ring, Chief, Reactor Projects Branch 1

ITEMS OPENED, CLOSED, AND DISCUSSED

Opened

05000254/2005007-01; 05000265/2005007-01	URI	Downgrade of Relief Valves from Category I Environmental Qualification (EQ) to Category II EQ Components
05000254/2005007-02; 05000265/2005007-02	URI	Lack of a Design Analysis Evaluating Secondary Fire Effects of Non-Fused 120 VAC Control Circuitry on the Plants' Fire Protection Safe Shutdown Analysis

Opened and Closed

None.

Discussed

None.

LIST OF DOCUMENTS REVIEWED

The following is a list of licensee documents reviewed during the inspection, including documents prepared by others for the licensee. Inclusion on this list does not imply that NRC inspectors reviewed the documents in their entirety, but rather that selected sections or portions of the documents were evaluated as part of the overall inspection effort. Inclusion of a document in this list does not imply NRC acceptance of the document, unless specifically stated in the inspection report.

IR02 Evaluation of Changes, Tests, or Experiments (71111.02)

10 CFR 50.59 Screenings

QC-S-2003-0177; QCOA 0300-02; Inability to Drive a Control Rod: Control Rod Stuck; Revision 0

QC-S-2003-0191; Traveling Screen High DP; Revision 0

QC-S-2003-0193; Control Room Ventilation System; Revision 0

QC-S-2003-0208; HPCI Automatic Initiation, HPCI System Manual Start-up (Injection/Pressure Control), HPCI Local Manual Operation; Revision 0

QC-S-2003-0252; QC TS Bases Change QC-BAS-03-006; Bases Change to Clarify Bases Tables B 3.8.7-1 and B 3.8.7-2; Revision 0

QC-S-2004-0012; RHR Pump A High Seal Leakage, RHR Pump B High Seal Leakage; RHR Pump C High Seal Leakage; RHR Pump D High Seal Leakage; Revision 0

QC-S-2005-0041; Modify High Level Feedwater and Turbine Trip; Revision 2

QC-S-2004-0067; Removal of Damper 2-5772-21A Supply Air to Annulus Region; Revision 0

QC-S-2004-0070; UFSAR-03-R8-041, QC-TRM-04-001; UFSAR & TRM Changes for RWCU and HPCI Valve Stroke Time Changes per EC 345237; Revision 0

QC-S-2004-0130; Defeating RCIC Suction Automatic Transfer to Torus; Revision 0

QC-S-2004-0151; Tech Spec Bases Change Request #QC-BAS-04-003, UFSAR Change # UFSAR-03-R8-048; RCIC Minimum Flow Valve Function Description Change to UFSAR and Tech Spec Bases; Revision 0

QC-S-2004-0152; QC-BAS-04-004; Technical Specifications Bases Change to Eliminate Reference to Valve Opening Time for the Reactor Building to Suppression Chamber Vacuum Breakers; Revision 0

QC-S-2004-0157; TRM Revision / QCTS 0340-07; Revise Unit 2 Turbine Bypass Valve Opening Time Profile; Revision 0

QC-S-2004-0275; Calculations QDC-8300-E-0966, Rev. 2 and QDC-8300-E-1420, Rev. 0 for 125 VDC Battery Testing; Revision 0

QC-S-2005-0106; EC 355532; Change Alignment of RCIC Pump Discharge Valves MO2-1301-48 and MO 2-1301-49; Revision 0

QC-S-2005-0106; TRM Change QC-TRM-05-008; TRM Change Package to Revise TRM 3.3.b Associated TRM Bases; Revision 0

QC-S-2005-0120; Allow For Controlled Air In-Leakage to the Off-Gas System, Revision 0

10 CFR 50.59 Evaluations

QC-E-2003-003; EC343934; Install Temporary Jumper to Remove Temperature Switch (TS) 1-0261-18A From Unit 1 "A" String Of Main Steam Tunnel High Temperature Trip Logic; Revision 0

QC-E-2004-001; Replacement of Target Rock PORVs with Dresser ERVs; Revision 0

QC-E-2004-003; EC21930; Unit 1 Reactor Recirculation Control System and Jet Pump Instrumentation Digital Upgrade (RRCS); Revision 0

QC-E-2004-006; Reactor Building Opening and Replacement Siding; Revision 0

QC-E-2005-005; Evaluation for the Cumulative Foreign Material Located in the U1 and the U2 Reactor vessel; Revision 0

QC-E-2005-007; Install Temporary Jumper Across Main Steam Line Tunnel High Temperature Switch; Revision 0

IR17 Permanent Plant Modifications (71111.17B)

Modifications

ECP 23888; Replace HPCI Orifice 2301-63C With Larger Diameter Orifice; Revision 0

EC 333021; Installation of Unit 2 SBLC Pump Suction Pipe Temperature Indication; Revision 0

EC 336482; Reconfigure the Motor Control Circuit for the 2B SBLC Pump; Revision 0

EC 342782; Reactor Feed Pump Oil Pressure Trip Logic Modified to 2 Out of 2; Revision 0

EC 347666; Process a Setpoint Change to Lower the Current Setpoint for Automatically Opening the AO 2-3401 Valves; Revision 0

EC 348796; Add Thermostatically Controlled Bypass Dampers to the CR HVAC B Train;
Revision 0

Other Documents Reviewed During Inspection

Corrective Action Program Documents Generated As a Result of Inspection

AR 00381760; The 10CFR50.59 QC-E-2004-003 Had Contradicting Information;
October 4, 2005

AR 00382283; TMOD 355532 Requires Enhancement; October 5, 2005

AR 00382437; UFSAR Discrepancy; October 5, 2005

AR 00382441; Potential UFSAR Discrepancy; October 5, 2005

AR 00382443; 50.59 Evaluation Could Not Be Found; October 5, 2005

AR 00382452; Issues with TMOD 355532; October 5, 2005

AR 00382847; MCC Control Power Transformer Circuit Issues; October 6, 2005

AR 00382862; UFSAR to Include Requirements for RB to Torus Vacuum Breakers;
October 6, 2005

AR 00382920; RWCU Isolation Valve Stroke Time Increase Evaluation Problem;
October 6, 2005

AR 00386837; EQ Requirements for Unit 2 ERVs; October 17, 2005

Corrective Action Program Documents Reviewed During the Inspection

AR 00148786; Tech Spec Bases 3.6.1.7 Statement Discrepancy; March 12, 2003

AR 00188260; Indications of FME Migration on 1B Recirculation Pump; November 26,
2003

AR 00188333; Foreign Material Evaluation for Dryer Pieces (Lost Parts); November 26,
2003

AR 00222314; U-1 SBLC Suction piping not maintaining temperature; May 19, 2004

AR 00236657; Cooling Coil Drip Pan Filled With Water - "B" Control HVAC; July 16,
2004

AR 00287210; Non-Compliance with WC-MW-8002 for Switchyard Work; January 3,
2005

AR 00345964; Temporary AMS Setups in the Plant Have Become Permanent; June 14, 2005

AR 00345968; Permanent Power to Heat Trace Is Improperly Supplied by an Extension Cord; June 14, 2005

AR 00345969; EHC Oil Purifiers Should Be Considered for Permanent Installation; June 14, 2005

Calculations

QC-030-M-002; RWCU Line Break P/T Analysis; Revision 000B

QDC-0600-I-1158; Reactor Water level Control System Setpoints and Parameters - Unit 2; Revision 1C

QDC-1600-M-1073; Design Comparison of Dresden to Quad Cities Containment and Vacuum Breakers; Revision 0

QDC-8300-E-0966; Development of a Modified Performance Test Duty Cycle for Unit 1 125 VDC Battery; Revision 0

Drawings

M-34 Sheet 1; Diagram of Pressure Suppression Piping; Revision BB

M-89 Sheet 1; Diagram of Reactor Core Isolation Cooling (RCIC) Piping; Revision AT

Procedures

AD-AA-101; Processing of Procedures and T&RMs; Revision 14

CC-AA-102; Design Input and Configuration Change Impact Screening; Revision 10

CC-AA-103; Configuration Change Control; Revision 9

CC-AA-103-2001; Setpoint Change Control; Revision 2

CC-AA-104; Document Change Requests; Revision 8

CC-AA-107; Configuration Change Acceptance Testing Criteria; Revision 4

CC-AA-107-1001; Post Modification Acceptance Testing; Revision 0

CC-MW-103-1001; Configuration Change Control Guidance; Revision 4

ER-AA-321; Administrative Requirements for Inservice Testing; Revision 5

LS-AA-104; Exelon 50.59 Review Process; Revision 4

LS-AA-104-1001; 50.59 Review Coversheet Form; Revision 2

LS-AA-104-1002; 50.59 Applicability Review Form; Revision 2

LS-AA-104-1003; 50.59 Screening Form; Revision 1

LS-AA-104-1004; 50.59 Evaluation Form; Revision 2

LS-AA-107; UFSAR Update Procedure; Revision 2

SM-AA-300; Procurement Engineering Support Activities; Revision 0

SM-AA-300-1001; Procurement Engineering Process and Responsibilities; Revision 5

QCAN 901(2)-6 F-7; Reactor Feed Pump Auto Trip; Revision 11

QCGP 3-1; Reactor Power Operations; Revision 46

QCOA 0300-02; Inability to Drive a Control Rod: Control Rod Stuck; Revision 12

QCOP 0202-03; Reactor Recirculation System Flow Controller Operation; Revision 16

QCOP 0600-21; Operation of the Feedwater Level Control System; Revision 10

QCOP 1100-03; Standby Liquid Control Tank Draining; Revision 8

QCOP 5750-09; Control Room Ventilation System; Revision 39

QCOS 1600-10; Torus Vacuum Breaker Manual Operability Test; Revision 12

QCOS 1600-14; Pressure Suppression System Power Operated Valve IST Testing; Revision 20

QCOS 1600-53; Drywell Average Air Temperature Monitoring; Revision 5

QOM 1-3200-01; U1 Feedwater Valve Checklist; Revision 13

Chart of Drywell Temperatures January to October 2005

Miscellaneous Documents

Letter from CYGNA to Commonwealth Edison; Quad Cities UFSAR Rebaseline Project;

UFSAR Section 10.4 Issue for CEC Co Concurrence; December 9, 1991

Letter from NRC to Dennis L. Farrar; Plant-Specific ATWS Review Guidelines and Implementation Schedule; December 17, 1986

Letter from NRC to Henry E. Bliss; Compliance with ATWS Rule 10 CFR 50.62 Relating to Alternate Rod Injection (ARI) and Reactor Coolant Recirculation Pump Trip (RPT) Systems for Dresden Units 2 and 3 and Quad Cities Units 1 and 2; November 8, 1988

EC 345237; Review of Design Required Stroke Times for DC Powered MOVs; February 26, 2004

EC 354934; Cumulative Effects of Foreign Material (FM) on the Reactor Vessel and Connecting Systems; Revision 2

TCCP No. EC 355532; Change Alignment of RCIC Pump Discharge Valves MO2-1301-48 and MO 2-1301-49; Revision 0

TIC-928; Work Order Package 00670869; Reduce the Auto Open Current Setting for the AO 2-3401 Valve (Modification Test for EC 347666); April 19, 2004

TIC-1097; Control Room Train B Ventilation Ductwork Modification Test/ (Modification Test for EC 348796); January 7, 2005

TIC-1195; Modification Test for U1 RFP Oil Pressure Trip Logic Modified to 2 Out of 2 (Modification Test for EC 342782); April 14, 2005

Completed Surveillance Test on Unit 1; QCOS 1600-10; Torus Vacuum Breaker Manual Operability Test; August 26, 2005

Completed Surveillance Test on Unit 2; QCOS 1600-10; Torus Vacuum Breaker Manual Operability Test; June 27, 2005

Results of Stroke Times from Completed Surveillance Test on Unit 1; QCOS 1600-14; Pressure Suppression System Power Operated Valve IST Testing; August 17, 2005

Results of Stroke Times from Completed Surveillance Test on Unit 2; QCOS 1600-14; Pressure Suppression System Power Operated Valve IST Testing; September 24, 2005

LIST OF ACRONYMS USED

ADAMS	Agency-Wide Document Access and Management System
CFR	Code of Federal Regulations
CPT	Control Power Transformer
DRP	Division of Reactor Projects
DRS	Division of Reactor Safety
ERV	Electromagnetic Relief Valve

EQ	Equipment Qualification
IMC	Inspection Manual Chapter
IR	Inspection Report
NCV	Non-Cited Violation
NEI	Nuclear Energy Institute
NRC	Nuclear Regulatory Commission
PORV	Power Operated Valve
PRA	Probabilistic Risk Assessment
SBLC	Standby Liquid Control
SDP	Significance Determination Process
VAC	Volts Alternating Current
VDC	Volts Direct Current
UFSAR	Updated Final Safety Analysis Report
URI	Unresolved Item